OREGON STATE TRIGA® REACTOR OPERATING PROCEDURES

OSTROP 18

Procedures for the Approval and Use of Reactor Experiments

Appendix A

Procedures for Irradiating a Sample in the Oregon State TRIGA[®] Reactor

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Procedures for Irradiating a Sample in the Oregon State TRIGA[®] Reactor

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OSTROP 18

Appendix A

Procedures for Irradiating a Sample in the Oregon State TRIGA®

Reactor

I. OSTR PROCESS, LICENSE, AND ENCAPSULATION LIMITATIONS

A. Process Overview

This document explains the proper method for requesting an irradiation in the Oregon State University TRIGA[®] Reactor (OSTR) located within the OSU Radiation Center. The following is a brief overview of what is required to perform an irradiation:

- 1. Contact the Radiation Center Director, Reactor Administrator or Reactor Supervisor to help you through this process.
- 2. If this is the first time you have performed an irradiation, please complete and submit the Project Application Form (Figure 1) to either the Director or Reactor Administrator of the Radiation Center. The Project Application Form is a generic way Radiation Center Staff collects information on the experimenter and irradiations. This information goes into the Radiation Center's project database and is used to track a project, create billing information, and to provide information for our Annual Report.
- 3. Prepare the samples in accordance with the encapsulation requirements described in this document.
- 4. Complete the required Irradiation Request (IR) forms. Procedures for filling out and examples of IR forms are given later in this document.
- 5. Mail or otherwise submit the samples and IR forms to the Reactor Administrator for irradiation. Samples and IR forms should be submitted 24 hours in advance of any scheduled irradiation. Also include a copy of the license to possess radioactive material from the appropriate licensing agency if a current copy is not on file at the Radiation Center.

B. OSTR Reactor Experiments

1. An experiment is a specifically described and approved use of the OSTR. Determination of which experiment your irradiation should be performed

under will be made by the Reactor Supervisor. The currently approved reactor experiments are:

- A-1 Normal TRIGA[®] Operations
- B-3 Irradiation of Materials in the Standard OSTR Irradiation Facilities
- B-29 Reactivity Worth of Fuel
- B-31 TRIGA[®] Flux Mapping
- B-36 Irradiation of Fissionable Materials in the OSTR
- 2. The description of a specific experiment may include additional limitations and more severe restrictions than those given below. The experimenter is obligated to be familiar with and comply with all limitations on a given experiment in addition to these general limitations. Any variations from these limitations or experiments must be approved by the OSTR Reactor Operations Committee.

C. OSTR License Limitations

- 1. The radionuclides and quantities to be produced for ultimate release shall not exceed the types and amounts authorized to the experimenter on an applicable license.
- 2. The steady state power level of the reactor shall not exceed 1100 kW thermal. (Administrative limit is 1000 kW.)
- 3. The reactivity inserted for pulse operation shall not exceed \$2.30. (Administrative limit is \$2.25.)
- 4. In-core experiments shall not occupy more than a single fuel element position.
- 5. The reactor shall not be operated at power levels exceeding 1 kW with a core lattice position vacant, except for positions on the periphery of the core assembly.
- 6. Non-secured experiments shall have reactivity worth less than \$0.50.
- 7. The total reactivity worth of all experiments shall not exceed \$2.30.
- 8. Explosive materials in quantities greater than 25 mg shall not be irradiated.
- 9. Explosive materials in quantities less than 25 mg may be irradiated provided the pressure produced upon detonation of the explosive has been

calculated and/or experimentally demonstrated to be less than half the design pressure of the container housing the explosives.

- 10. Where the possibility exists that the failure of an experiment (except fueled experiments) could release radioactive gases or aerosols to the reactor bay or the unrestricted area, the quantity and type of material to be irradiated shall be limited such that the airborne radioactivity in the reactor bay or the unrestricted area shall not result in exceeding the applicable dose limits in 10 CFR 20.
- 11. Irradiation of the following materials shall require specific prior review and approval by the Reactor Operations Committee, including the Reactor Supervisor, the Senior Health Physicist, and the Reactor Administrator, even when such an irradiation appears to fall generally within the scope of an approved reactor experiment.
 - a. Highly flammable organic solids and solvents, or other highly flammable materials.
 - b. Any substance known or likely to exhibit characteristics due to irradiation that would require special safety precautions. Such characteristics include, but are not limited to, excessive gas buildup in sample containers, unusually high radiation dose rates, and the release of airborne radioactivity.
 - c. Anything containing mercury or gallium.

D. OSTR Encapsulation Limitations

1. Specifically-Approved Minimum Encapsulation Methods

The methods of encapsulation detailed in Tables 18.1 through 18.4 below have been found by testing and experience to provide satisfactory containment for the specified sample forms. Designated containments are applicable only to the specific irradiation facilities, the physical forms of materials, and the corresponding irradiation times shown in the tables. Proposed use of encapsulations not specified in the tables will be evaluated under "Other Encapsulation Methods." All polyethylene vials, except those designated as "vented," and all Nalgene liquid scintillation bags will be heat-sealed. Other containments will be closed according to the specifications given in the tables. Except for the thermal column and beam ports, containments specified in the tables are in addition to the containment provided by the sample tube or rabbit tube used to hold encapsulated samples during irradiation.

2. Other Encapsulation Methods

Various other methods of encapsulation may also be suitable. However, before other methods may be used by experimenters they must be approved by the Reactor Supervisor and the Senior Health Physicist. This approval will be based on actual testing of the proposed containment, or on other documented evaluations which conclude that a proposed encapsulation will provide an acceptable degree of containment when compared to encapsulations currently approved for the type of material involved, the irradiation time, and the irradiation facility. The Reactor Supervisor will maintain records of tests or other evaluations used to determine the containment capability and/or general suitability of proposed encapsulations. The approval of encapsulations by the Reactor Supervisor and the Senior Health Physicist will be documented by their signatures on the applicable Irradiation Request.

3. ROC Approved Reactor Experiments

If a specific experiment has been approved by the ROC then that experiment and its requirements regarding encapsulation, or lack of it, will take precedence over the encapsulation requirements described here.

4. Generic Statement on the Relative Adequacy of Various Encapsulations

This procedure provides guidance on the minimum acceptable encapsulations for each of the various OSTR irradiation facilities and requires documentation for deviations from the standard encapsulations given. The purpose of this statement is to allow a more rigorous encapsulation to be used in any of the OSTR irradiation facilities without further approval other than the signatures of the Senior Health Physicist and the Reactor Supervisor on the Irradiation Request Form. A more rigorous encapsulation is defined as the use of flame sealed quartz or sealed aluminum containers instead of polyethylene containers. Many years of experience and actual use have shown that there are no limitations to the use of sealed quartz containers or aluminum containers for irradiating samples in the OSTR. Therefore, substituting these containers in cases where polyethylene is allowed only increases the level of confidence in the containment integrity.

II. COMPLETING THE IRRADIATION REQUEST INFORMATION SHEET

A. Why an Information Sheet is Needed

The Irradiation Request (IR) Information Sheet shall be completed by the experimenter each time an Irradiation Request is submitted. The Irradiation Request Information Sheet is divided into two columns. The experimenter will complete the left column. Radiation Center staff completes the right column. The form allows our staff to document the necessary checks prior to any sample irradiation. While some of the information requested helps the Radiation Center track your sample in an efficient manner, most of the information is needed to meet federal requirements enforced by the Nuclear Regulatory Commission. Essentially, we need to know what is going into the reactor, what is coming out, and who authorized the irradiation. It also documents that the experimenter is aware of what is in the sample and that it conforms to our requirements.

The IR is an important document for OSTR staff because it is the first line of defense for performing irradiations in a safe and responsible manner. By signing the IR, the experimenter is stating the follow conditions have been met:

- 1. The experimenter is familiar with the general scope, requirements and limitations of the reactor experiment specified on the IR. All personnel submitting IRs under their signed authorization are also familiar with the necessary details of the applicable reactor experiment.
- 2. Irradiation Requests submitted meet the conditions of the indicated reactor experiment.
- 3. The experimenter is authorized by a currently valid OSU or other appropriate radioactive materials license to perform the activities covered by the indicated reactor experiment and to possess any radionuclides generated in the course of the experiment.
- 4. All individual items or samples to be irradiated in the reactor, and all major radionuclides to be produced as a result of the irradiation, have been clearly and accurately listed on the IR. No items may be irradiated unless they are listed on the IR.
- 5. Changes may be made to an IR after one or more of the required approval signatures have been obtained only with the approval of the Reactor Supervisor and the Senior Health Physicist. The Reactor Supervisor and

the Senior Health Physicist will indicate approval of a change by initialing and dating the change.

- 6. The following items require notifying the Reactor Supervisor and Senior Health Physicist before irradiation:
 - a. Explosive materials, such as gunpowder, TNT, and nitroglycerin;
 - b. Hazardous organic compounds, solvents or highly flammable organics, such as gasoline;
 - c. Special nuclear materials, such as enriched uranium; no natural uranium, depleted uranium, or natural thorium in excess of 1 mg of each substance per individual sample;
 - d. Samples known or likely to exhibit potentially hazardous characteristics due to irradiation, such as excessive gas build-up in irradiation containers; and
 - e. Materials known or likely to create undesirable conditions, such as unusually high dose rates or airborne radioactivity.

These materials to be irradiated must be permissible within the limits of the reactor Technical Specifications and the limits specified within the designated reactor experiment.

B. General Information Block

1. Institution

Please enter the name of the university, company, institute, or other for whom the irradiation will be performed.

2. Individual User

Please enter the name of the individual for whom the irradiation will be performed.

3. Project Name

Please enter the name of the project under which this irradiation will be performed. This should be the same project name as that used for the initial project application. The project name helps the Radiation Center Staff make sure that samples to be irradiated correspond or are otherwise appropriate for the approved project and reactor experiment.

C. Payment Information Block

1. (OSU) Index Number

For projects by experimenters at OSU, mark this box and enter the index number to charge the irradiation costs against.

Mark this box for irradiations performed for experimenters not at OSU. Enter the purchase order number to be charged against.

3. Unfunded (Not Applicable)

Mark this box if the irradiation is unfunded (*pro bono*) or funding is not applicable. Radiation Center staff must approve *pro bono* irradiations in advance.

D. OSU Radiation Safety Approval Block

1. Not Required

Mark this box if you are not a researcher/experimenter at OSU or the material remains on license R-106. OSU Radiation Safety approval is only required for experimenters affiliated with OSU performing an irradiation that produces material that moves to the State of Oregon license. It is also not required for irradiations performed under Experiment A-1 because this experiment involves irradiations that do not produce any radioactive material.

2. Radiation Safety Approval Number

Check this box if you are a researcher/experimenter at OSU. To obtain an approval number, simply contact OSU Radiation Safety (737-2227) and ask for one. They will ask for the approximate date of irradiation, the isotopes to be produced, and the total activity at the time of transfer. The isotopes produced and activity produced will also be entered on the Sample Information Form. After checking against your Radiation Use Authorization (RUA), they will issue the approval number. Enter the approval number in this block on the Irradiation Request Information Sheet.

E. Irradiation Information Block

1. Power/Time

Mark this box if you know the power and time needed for the irradiation. Enter the reactor power in units of kW and the irradiation time in units of hours. Mark this box if you know the total fluence needed for the irradiation. Enter the total fluence in units of neutrons cm⁻². Radiation Center staff will calculate the needed reactor power and time for you based on the desired irradiation facility and the location of the sample in the encapsulation.

3. 1 MeV Equivalent Fluence

Mark this box if this irradiation is for analysis of radiation damage to materials as defined in ASTM E722-14. Enter the total fluence in units of 1 MeV equivalent neutrons cm^{-2} .

4. Reactor Irradiation Facility

Enter the reactor facility in which you would like the irradiation to take place. Authorized Facilities include:

CLICIT (Cadmium-Lined In-Core Irradiation Tube): B-Ring or F-Ring ICIT (In-Core Irradiation Tube): F-Ring only Rotating Rack (Lazy Susan) Thermal Column Pneumatic Transfer System (Rabbit) NRF (Neutron Radiography Facility) PGNAA (Prompt Gamma Neutron Activation Analysis Facility)

F. Encapsulation Block

Encapsulation is the primary means we have to prevent contamination from occurring both in and out of the reactor. Typically a facility will require both a primary and secondary encapsulation. Specific encapsulation requirements are listed in Tables 18.1 through 18.4 below. Examples of encapsulation descriptions include, but are not limited to, the following:

"Sealed quartz in aluminum TRIGA[®] tube" "Polyethylene TRIGA[®] tube" "Sealed aluminum TRIGA[®] tube" "Double encapsulated heat-sealed poly vial in polyethylene TRIGA[®] tube"

G. Brief Project Description Block

Enter a brief description of the project under which this irradiation will be performed. This should be the same project description as that used for the initial project application. The project description may seem redundant (similar to the project name), but it helps the Radiation Center Staff make sure that samples to be irradiated correspond or are otherwise appropriate for the approved project and experiment.

H. Experimenter Signature Block

The experimenter must sign and date here. By signing, the experimenter is stating that the irradiation follows the guidelines set forth in *Procedures for Irradiating Samples in the Oregon State TRIGA*[®] *Reactor* (this procedure).

1. Experimenter License Number

Mark this box if your work falls under the jurisdiction of OSU Radiation Safety. Also please fill in your RUA number.

2. Other

Mark this box if you are not a researcher/experimenter at OSU. Additionally, please enter the appropriate radioactive material license number.

III. COMPLETING THE SAMPLE INFORMATION FORM

A. Process Overview

The intent of the sample information forms is to collect information from the experimenter on the composition of the samples. There is a Standard Sample Information Form and a Pneumatic Transfer Sample Information Form. The Pneumatic Transfer Sample Information Form is different because it assumes that the sample is immediately transferred to the experimenter following the irradiation. There are three important reasons for collecting this information:

1. Reactivity Effects

Samples containing elements like U, B, and Cd can have dramatic effects upon the core. In some cases, more neutrons are produced than anticipated (e.g., U, Pu and Th) and in other cases, more neutrons are absorbed than is desirable (e.g., B and Cd). These can cause changes in neutron flux and ultimately, core reactivity (a measure of changes in neutron population across the entire core)

2. Radiation Safety

Even small samples can potentially be a radiological concern. Typically, we handle each irradiated sample twice; once when the sample is removed from the reactor and again when it is prepared for shipping or analysis. It

is important that we understand what may be produced such that we can anticipate samples that create significant radiation fields. For example, if a sample produces a significant quantity of a radioisotope with a half-life on the order of 6 hours, we may choose to time the irradiation to end on a Friday afternoon so that we remove the sample from the reactor on Monday morning.

3. License Responsibility

We are required by law to verify that each institution we send samples to is licensed (authorized by the appropriate regulatory agency) to possess the isotopes and activity found in each shipment. This form allows us to verify, in advance, that the license will cover the types and amounts of radioisotopes produced for the irradiation.

The experimenter is responsible for entering all applicable information into the shaded section of the form. Below is a brief description of the information that is necessary for the experimenter to properly complete the form.

B. Sample Number Column

Enter the sample number here. A sample can be defined as anything that requires encapsulation. Here are a couple of examples: Fission track users will typically have 20-30 different specimens acquired from different geological formations. We (OSU) would consider this to be one sample because all 20-30 specimens will easily fit into one polyethylene sample tube (the encapsulation method). For INAA irradiations, there may be two 2/5-dram vials in one 2-dram vial. We (OSU) would consider this to be two samples, each requiring an entry into the form. For material science or ${}^{39}\text{Ar}/{}^{40}\text{Ar}$ irradiations, each sample would be defined by what fit into the quartz tube.

C. Physical Form Column

Enter the physical form of the sample. For most samples solid, liquid, gas or powder will suffice. The physical form will, in many cases, determine the type of encapsulation required.

D. Chemical Form Column

Enter the chemical form of the sample. Common examples include "NaCl", "apatite", "zircon", "metal foil", "ceramic", and "plastic".

E. Sample Amount Column

Enter the mass of each sample in units of grams.

F. Estimated Radioactivity at EOB

Enter the estimated radioactivity of the sample at the end of bombardment (EOB). The equation for determining the activity of an isotope after an irradiation is:

$$A(t) = \frac{N_{av}m}{AW} \left(\frac{\sqrt{\pi}}{2}\sigma_{th}\phi_{th} + RI\phi_{epi}\right) \left(1 - e^{-\frac{(\ln 2)t}{t_{1/2}}}\right)$$

Where:

A= Activity of radioactive progeny of isotope irradiated (Bq) (3.7E7 Bq = 1 mCi) N_{av} = Avogadro's Number (6.02E23 atoms mol⁻¹) m = Sample mass (g) AW = Atomic weight (g mol⁻¹)

 σ_{th} = Neutron absorption cross section for thermal neutrons (cm²)

 ϕ_{th} = Thermal neutron flux (neutrons cm⁻² s⁻¹)

RI = Resonance Integral for epithermal neutrons (cm²)

 ϕ_{epi} = Epithermal neutron flux (neutrons cm⁻² s⁻¹)

t = Irradiation time (s)

 $t_{1/2}$ = Half-life of the radioactive progeny of the isotope irradiated (s)

Numbers for σ_{th} , RI, $t_{1/2}$, and AW can be obtained from a number of sources, the most common being the Chart of the Nuclides. Values for the thermal and epithermal fluxes can be obtained by contacting the Radiation Center directly. The following is an example of how to calculate the activity of 66 Cu from the reaction 65 Cu + n $\rightarrow {}^{66}$ Cu:

$$AW = 64.928$$

m = 0.00315 g

$$\sigma_{th} = 2.17 \text{ b} = 2.17 \text{ X} 10^{-24} \text{ cm}^2$$

RI = 2.2 b = 2.2X10^{-24} \text{ cm}^2
 $t_{1/2} = 5.10 \text{ min}$
 $\lambda = \frac{\ln 2}{5.10 \text{ min}} = 0.136 \text{ min}^{-1}$
 $t = 5 \text{ min}$
 $\phi_{th} = 1X10^{10} \text{ cm}^{-2} \text{s}^{-1}$
 $\phi_{epi} = 3X10^9 \text{ cm}^{-2} \text{s}^{-1}$

$$A_{66}_{Cu} = \left(\frac{6.02 \times 10^{23} (0.00315)}{64.928}\right) \left[\frac{\sqrt{\pi}}{2} 2.17 \times 10^{-24} \left(10^{10}\right) + 2.2 \times 10^{-24} \left(3 \times 10^{9}\right)\right] \left(1 - e^{-0.136(5)}\right)$$
$$A_{66}_{Cu} = 3.72 \times 10^{5} dps = 10.1 \, \mu Ci$$

G. Major Radionuclides at EOB

List the major radionuclides that will be present in the sample at EOB. This information allows the Radiation Center staff to identify samples that may create high radiation fields if immediately pulled. The most common example is samples that produce a large amount of ²⁴Na. We typically time these irradiations such that the irradiation terminates on Friday afternoon and the sample is allowed to decay over the weekend before pulling the sample from the reactor Monday morning.

H. Estimated Radioactivity at Transfer (N/A for Pneumatic Transfer Sample Form)

Enter the activity of the sample after a decay period of two weeks. This is the standard time we allow samples to decay before packaging them for shipping. The reason we wait is twofold. First, the activity and dose rate on the package surface for most samples would be much higher than allowed by federal regulations to transport. Second, this is the best way to maintain our personnel dose as low as reasonably achievable due to the number of samples handled by our staff. Only in unusual circumstances will we prepare a shipment before the two-week decay period has elapsed. The equation for radioactive decay is:

$$\mathbf{A} = \mathbf{A}_{\mathbf{0}} \mathbf{e}^{-\left(\frac{\ln 2}{t_{1/2}}\right)t}$$

where

А	=	Activity of isotope after decay
Ao	=	Original activity of the isotope
t	=	Time of decay
t _{1/2}	=	half-life of isotope

Cadmium	Physical	Integrated	Minimu	m Containment
Covers	Form of Material	Power (MWh)	Primary	Secondary ¹
		< 25	Polyethylene Vial	Not required
	Stable Solid	25-35	Flame-sealed quartz or sealed container	Not required
Without Standard	bond	> 35	Flame-sealed quartz or sealed container	Not required
Cadmium Covers	Liquid,	< 25	≤ 2 dram polyethylene vial	\leq 4 dram polyethylene vial
Covers	Powder or Other Loose Solid Material	> 25	Flame-sealed quartz or welded aluminum container	Approved sealant on an aluminum TRIGA [®] tube
	Stable Solid	≤1	≤2/5 dram polyethylene vial, or aluminum foil wrap inside cadmium cover inside ≤□4 dram polyethylene vial	Not required
With		Stable Solid	stable Solid	Flame-sealed quartz or sealed aluminum container
Standard Cadmium Covers ⁽²⁾		<u>≤</u> 35	Aluminum foil wrap inside cadmium cover inside sealed aluminum container	Not required
	Liquid, Powder or Other Loose Solid Material ⁽³⁾	≤1	≤2/5 dram polyethylene vial inside Cd container	4 dram polyethylene vial outside the Cd cover

Table 1- Minimum Encapsulation for the Rotating Rack (Lazy Susan)

Footnote:

- (1) The standard carriers for LS irradiations are the poly or aluminum TRIGA[®] Tube for irradiations less than 25 hours and the aluminum TRIGA[®] tube for irradiations greater than 25 hours. These tubes are not considered secondary containment.
- (2) The standard cadmium cover referenced in Table 18.1 consists of a small cadmium box and cover which is capable of holding a conventional 2/5 dram or 2/27 dram polyethylene vial.
- (3) Cd covered irradiations exceeding 1 MWhr should be conducted within the CLICIT.

Cadmium CoversPhysical Form of MaterialIntegrated Power (kWh)Minimum Primary		Integrated	Minimum Containment				
		Secondary					
	Stable Solid	≤1000	≤ 2 dram polyethylene vial	Not required			
Without Cadmium Covers	Liquid, Powder or Other Loose Solid Material	≤1000	$\leq 2/5$ dram polyethylene vial	≤2 dram polyethylene vial			
With	Stable Solid	≤100	≤2/5 dram polyethylene vial, or aluminum foil wrap inside cadmium cover inside ≤2 dram polyethylene vial	Not required			
Cadmium Covers ⁽¹⁾	Liquid, Powder or Other Loose Solid Material	≤100	$\leq 2/5$ dram polyethylene vial	≤2 dram polyethylene vial			

Table 2 - Minimum	Encapsulation	for the Pneumati	c Transfer	• Tube (Rabbit)
	Lincupsulation	tor the i neuman	c if anoici	

Cadmium	Physical	Minir	num Containment
Covers	Form of Material	Primary	Secondary
	Stable Solid	Polyethylene TRIGA [®] tube or other equivalent plastic containment	Not required
With or Without Cadmium Covers ⁽¹⁾	Liquid, Powder or Other Loose Solid Material	Sealed polyethylene container	Polyethylene container
	Stable Gas	Impervious container known to be leak-tight or sealed in a conservative manner. Gas samples shall not be compressed or pressurized.	Sealed plastic bag, or equivalent (bag may be heat sealed or zip-lock)

Table 3 - Minimum Encapsulation for the Thermal Column and Beam Ports⁽²⁾

Footnote:

(1) Cadmium covers referenced in the above table have no specific shape or form requirements other than those imposed by the dimensions of the primary and secondary containment.

(2) Radiography samples, at the discretion of the Senior Health Physicist and the Reactor Supervisor, may be exempt from this requirement

Cadmium	Physical Form	Minimum Containment				
Covers	of Material	Primary	Secondary			
	Stable Solid	Flame-sealed quartz or sealed aluminum container	Not required			
Without Cadmium Covers	Liquid, Powder or Other Loose Solid Material	Flame-sealed quartz or sealed aluminum container	Sealed aluminum container			

Table 4 -	Minimum	Encapsulatio	n for the In-	-Core Irra	diation Tu	bes (ICIT/CLICIT)
		Lincapsulatio		COLCINA		

PROJECT APPLICATION FORM

This form will be used to create a project in our projects database. Please fill out the following information fields.

Principal Investigator (PI)

First Name:

Last Name (Surname/Family Name):

Email Address:

Organization Name:

Address Line 1:

Address Line 2:

City:

State:

Postal Code:

Country:

Phone Number:

Fax Number (if applicable):

Invoicing

Invoice Email (if different from PI):

Invoice First and Last Name (Surname/Family Name):

Invoice Address Line 1:

Invoice Address Line 2:

Invoice City:

Invoice State:

Invoice Postal Code:

Invoice Country:

<u>Project</u>

Project Name:

Project Description (3 sentences maximum):

Figure 1 – Project Application Form

OSU TRIGA IRRADIATIO INFORMAT	A REACTOR DN REQUEST TON SHEET	IR #: Project #: Run #: Page of
GENERAL INFORMATION Institution:	RECEIPT OF RADIOACTIVE MATERIAL PRIOR TO IRRADIATION (Staff Use Only)	dB□
Individual User: Project Name:	Isotopes:	
PAYMENT INFORMATION OSU) Index Number: (Non-OSU) Purchase Order Number: Unfunded (Not Applicable)	Physical/Chemical Form:	
OSU RADIATION SAFETY APPROVAL	OPERATIONS STAFF APPROVAL (Staff Use Or	nly)
Radiation Safety Committee Approval Number:	Senior Health Physicist Signature:	Date: Date:
	Experiment Number:	
IRRADIATION INFORMATION	REACTOR TIME (Staff Use Only)	dB□
Facility General Reactor Use	Required Irradiation Time: [h	rs]
Reactor Power/Time: [kW] [hours]	Date	
□ Total Neutron Fluence: [n cm ⁻²]	Power (kW)	
□1 MeV Equivalent Fluence: [n cm ⁻²]	Start Time	
Remarks:	End Time	
	∆Time	
	Total Irradiation Time:	rs]
ENCAPSULATION	Time conversion (for CLOCIT irradiations):	
	Requested B1 CLICIT Time [h 1.8	rs]
	= F20 CLOCIT Equivalent Time[hrs	3]
BRIEF PROJECT DESCRIPTION	MATERIAL TRANSFER (Staff Use Only)	dB□
	Material transferred from License R-106 to:	
	ORE 90005 RUA #:	
	Other:	
	Total Radioactivity Transferred:	
EXPERIMENTER SIGNATURE	RAM Transfer Record Number:	
the experimenter for this IK declares that this irradiation requested falls under the guidelines of requirements given in OSTROP 18 Appendix A "Procedures for Irradiating a Sample in the Oregon State TRIGA Reactor."	Transfer Date: Transfer Time:	
Experimenter Signature Date	Shipped/received by:	
Experimenter License Number: 🔲 ORE 90005 RUA #:	Remarks:	
Other:		

Figure 2 - Information Request Information Sheet

			STAN (Shac	IRRA DARD SA led columns	ADIATION AMPLE IN must be comp	FORMA	CST TION E Exper	N FOF	RM r)	IR # Proj Run Page	ect #:	of
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Figure 3 - Standard Sample Information Form

IRRADIATION REQUEST PNEUMATIC TRANSFER SAMPLE INFORMATION FORM

(Shaded columns must be completed by the Experimenter)

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Figure 4 - Pneumatic Transfer Sample Information Form

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Ŭ	Jser Name											
	Facility:			TC irradiation calculation								
				Average sample Cl (cm) =								
Total	MWHR req	uired =		Required Irradiation flux =								
				Time of irradiation (hr) = N/A								
	Ti	me		Calculated values								
Date	Begin	End	Power	Delta	MWHR	Time remaining	S/D Time					

Figure 5 - Irradiation Time Tracking Form

OSU TRIGA IRRADIATIC INFORMAT	IR #:
GENERAL INFORMATION Institution: Bologna University Individual User: John Doe Project Name: Fission Track Project Name: GOSU) Index Number: (OSU) Index Number: Ø (Non-OSU) Purchase Order Number: T21354 Unfunded (Not Applicable)	RECEIPT OF RADIOACTIVE MATERIAL dB PRIOR TO IRRADIATION (Staff Use Only) N/A (Non-radioactive material) Isotopes:
OSU RADIATION SAFETY APPROVAL Structure of the second	OPERATIONS STAFF APPROVAL (Staff Use Only) Senior Health Physicist Signature: Date: Reactor Supervisor Signature: Date: Experiment Number:
IRRADIATION INFORMATION Facility: Thermal Column	REACTOR TIME (Staff Use Only) dB□ Required Irradiation Time:
ENCAPSULATION Poly TRIGA Tube	Remarks: Time conversion (for CLOCIT irradiations): Requested B1 CLICIT Time [hrs] x 1.8 = F20 CLOCIT Equivalent Time [hrs]
BRIEF PROJECT DESCRIPTION Use of fission track analysis to determine U content in the Rhine Gruben uplift. EXPERIMENTER SIGNATURE The experimenter for this IR declares that this irradiation requested falls under the guidelines of requirements given in OSTROP 18 Appendix A "Procedures for Irradiating a Sample in the Oregon State TRIGA Reactor." Experimenter Signature John Doe Date 7/20/18 Experimenter License Number: ORE 90005 RUA #:	MATERIAL TRANSFER (Staff Use Only) dB Material transferred from License R-106 to:

Figure 6 - Example Information Sheet for Fission Track Irradiations

OSU TRIGA IRRADIATIO INFORMATION Institution: Oregon State University Individual User: John Doe Project Name: Cross Section Calculation PAYMENT INFORMATION \$\text{Delta}(OSU) Index Number: ENE 333 \$(Non-OSU) Purchase Order Number: State	IR #: IR #: IR #: Project #: Project #: Run #: Page of PRIOR TO IRRADIACTIVE MATERIAL dB PRIOR TO IRRADIATION (Staff Use Only) N/A (Non-radioactive material) Isotopes: Activity: Physical/Chemical Form: Date: Date: Time:							
Unfunded (Not Applicable)	Receiver's signature: Rx Sup							
OSU RADIATION SAFETY APPROVAL Image: Description of the second	OPERATIONS STAFF APPROVAL (Staff Use Only) Senior Health Physicist Signature: Date: Reactor Supervisor Signature: Date: Experiment Number: Date:							
IRRADIATION INFORMATION Facility: Pneumatic Transfer	Date [] Power (kW) [] Start Time [] End Time [] Total Irradiation Time: []	dB□ rs]						
ENCAPSULATION 2/5 Dram Sealed Poly Vial inside 2 Dram Sealed Poly Vial	Remarks: Time conversion (for CLOCIT irradiations): Requested B1 CLICIT Time [h x 1.8 = F20 CLOCIT Equivalent Time [hr	rs] s]						
BRIEF PROJECT DESCRIPTION Determination of 198 Au thermal neutron cross section	MATERIAL TRANSFER (Staff Use Only) Material transferred from License R-106 to: ORE 90005 RUA #:	dB						
EXPERIMENTER SIGNATURE The experimenter for this IR declares that this irradiation requested falls under the guidelines of requirements given in OSTROP 18 Appendix A "Procedures for Irradiating a Sample in the Oregon State TRIGA Reactor." Experimenter Signature John, Doe Date 7/20/18 Experimenter License Number: I ORE 90005 RUA #: 99.9 I Other: Other:	Total Radioactivity Transferred:							

Figure 7 - Example Information Sheet for OSU Experimenter with Rabbit Irradiation

OSU TRIGA IRRADIATIO INFORMAT	IR #:						
GENERAL INFORMATION Institution: Portland College Individual User: John Doe Project Name: Ar/Ar Geochronology PAYMENT INFORMATION (OSU) Index Number:	RECEIPT OF RADIOACTIVE MATERIAL dB PRIOR TO IRRADIATION (Staff Use Only) \ \/\ \(Non-radioactive material) Isotopes:						
OSU RADIATION SAFETY APPROVAL Image: Strength Strengt Strengt Strengt Strength Strength	OPERATIONS STAFF APPROVAL (Staff Use Only) Senior Health Physicist Signature: Date: Reactor Supervisor Signature: Date: Experiment Number:						
IRRADIATION INFORMATION Facility: CLICIT (B1)	REACTOR TIME (Staff Use Only) dB□ Required Irradiation Time: [hrs] Date						
ENCAPSULATION Sealed Quartz inside Sealed Aluninum TRIGA Tube	Time conversion (for CLOCIT irradiations): Requested B1 CLICIT Time [hrs] x 1.8 = F20 CLOCIT Equivalent Time [hrs]						
BRIEF PROJECT DESCRIPTION <u>Ar/Ar</u> <u>analysis</u> for <u>age</u> <u>dating</u> of <u>geologic</u> <u>materials</u> EXPERIMENTER SIGNATURE The experimenter for this IR declares that this irradiation requested falls under the guidelines of requirements given in OSTROP 18 Appendix A "Procedures for Irradiating a Sample in the Oregon State TRIGA Reactor." Experimenter Signature <u>Johns</u> <u>Ose</u> Date 7/20/18	MATERIAL TRANSFER (Staff Use Only) dB Material transferred from License R-106 to:						

Figure 8 - Example Information Sheet for Ar/Ar Geochronology Irradiations

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L						Final Release	Survey							
Group							Uncorrect	ed Contact	Un	corrected @	1 ft		Estimated	
Number	Date		Time		Instrument / S	enal Number	(mk WO	(mR h ⁻¹)		(mR h ⁻¹)		BSCF	[6CEN]	
,										*** \(\)1		+	(niCi)	
1														
2												 		
3														
4														
												-		

Estimated Activity Transferred (mCi) =

